

September 1, 2005

LICENSEE: Nuclear Management Company, LLC

FACILITY: Monticello Nuclear Generating Plant

SUBJECT: SUMMARY OF A TELEPHONE CONFERENCE CALL HELD ON AUGUST 10, 2005, BETWEEN THE U.S. NUCLEAR REGULATORY COMMISSION AND NUCLEAR MANAGEMENT COMPANY, LLC (NMC) TO DISCUSS FOLLOW-UP ITEMS REGARDING THE SEVERE ACCIDENT MITIGATION ALTERNATIVES (SAMA) REQUEST FOR ADDITIONAL INFORMATION (RAI) RESPONSES FOR MONTICELLO NUCLEAR GENERATING PLANT (TAC NO. MC6441)

On August 10, 2005, the U.S. Nuclear Regulatory Commission (NRC or the staff) and its contractor from Information Systems Laboratory (ISL) conducted a conference call (teleconference) with representatives from NMC to discuss the Monticello Nuclear Generating Plant (Monticello) SAMA RAI responses (see ADAMS Accession No. ML052130197). The SAMA RAI responses were submitted to the NRC on July 27, 2005. The teleconference was held to clarify these RAI responses. Enclosure 1 provides a listing of the teleconference participants.

The NRC staff asked for clarification regarding RAI 1.d, 1.e, 2.b, and 5.b. NMC staff provided information to help clear up those questions. No staff decisions were made during the teleconference. Enclosure 2 contains an email response from NMC clarifying their response to RAI 5.b.

/RA/

Jennifer Davis, Project Manager
Environmental Section
License Renewal and Environmental Impacts Program
Division of Regulatory Improvement Programs
Office of Nuclear Reactor Regulation

Docket No.: 50-263

Enclosures: As stated

cc w/encls: See next page

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AUGUST 10, 2005**

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Date: 8/12/05 10:28AM
Subject: Supplement SAMA RAI 5b

To supplement Response to Request for Additional Information Regarding Severe Accident Mitigation Alternates for Monticello Nuclear Generating Plant (TAC NO. MC6441), dated July 27, 2005 (Accession Number ML052130197) and telecon on August 10, 2005.

NMC provides herein clarifying information for our response to NRC SAMA RAI 5.b regarding SAMA 37 (Manual Reactor Core Isolation Cooling (RCIC) Operation) and combined impact of recommended SAMAs:

Given a prolonged station blackout (SBO), SAMA 37 (Manual RCIC Operation) alone does not provide a success path for preventing core melt, because containment heat removal and containment venting are both unavailable. This eventually results in containment rupture due to overpressure, precluding operators from occupying the RCIC room to support manual operation of the system. In addition, RCIC would eventually deplete the condensate storage tank (CST) inventory and would overheat if torus water (saturated conditions) was used as the suction source.

For these reasons, RCIC only shifts risk dominant sequences from leading to core melt in 6 hours (2 hours after batteries deplete) to 20 hours (a few hours after containment ruptures or CST inventory is depleted). This has competing effects on risk. The delay allows an increased probability of recovering offsite power or power from an emergency diesel generator, which reduce risk. At the same time, it also increases risk because the additional time allows containment to heat up and pressurize.

The risk reduction gained by allowing an increased probability of power recovery (ending SBO) is far outweighed by the increase in risk due to shifting to more severe consequences of failing a pressurized reactor vessel into a pressurized containment at saturated conditions. However, SAMA 37 is highly effective at mitigating risk if other supporting SAMAs are also implemented to assure its viability and reliability. Those supporting SAMAs are summarized as follows:

SAMA 28 (Refill CST) establishes the use of the fire protection system to ensure RCIC has a long-term suction source.

SAMA 12 (Additional Diesel Fire Pump for Fire Service Water System) improves the reliability of the fire protection system, especially during SBO scenarios. This dramatically improves the suction source reliability for RCIC since CST is filled from fire protection system).

SAMA 2 (Enhanced DC Power Availability) provides power to solenoid valves, allowing containment to be vented during prolonged SBO

scenarios, preventing RCIC failure.
SAMA 36 (Divert Water from Turbine Building 931-Foot Elevation East)
improves RCIC reliability by preventing flooding of supporting batteries
before enough time is available to establish manual RCIC operation.

If you have any questions regarding this information please contact Jim
Holthaus, Monticello License Renewal, at (763) 295-1309.

James J. Holthaus

Monticello Nuclear Generating Plant License Renewal

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